



Pacific Northwest
National Laboratory
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Dosimetry Modeling at the Fuel-Water Interface

Presented to the:
**International High Level
Radioactive Waste Management
Conference**

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Introduction

- Dissolution of fuel is affected by oxidants and reductants:
 - These products are a function of:
 - » Initial radiolytic products formed
 - » Spatial distribution of these species
 - » Resulting chemistry among these products and fuel
 - Initial radiolytic products are a function of:
 - » Type of radiation
 - » Magnitude and energy of radiation
 - » Fuel pellet or grain size (surface-to-volume ratio)
 - » Fuel-to-water ratio

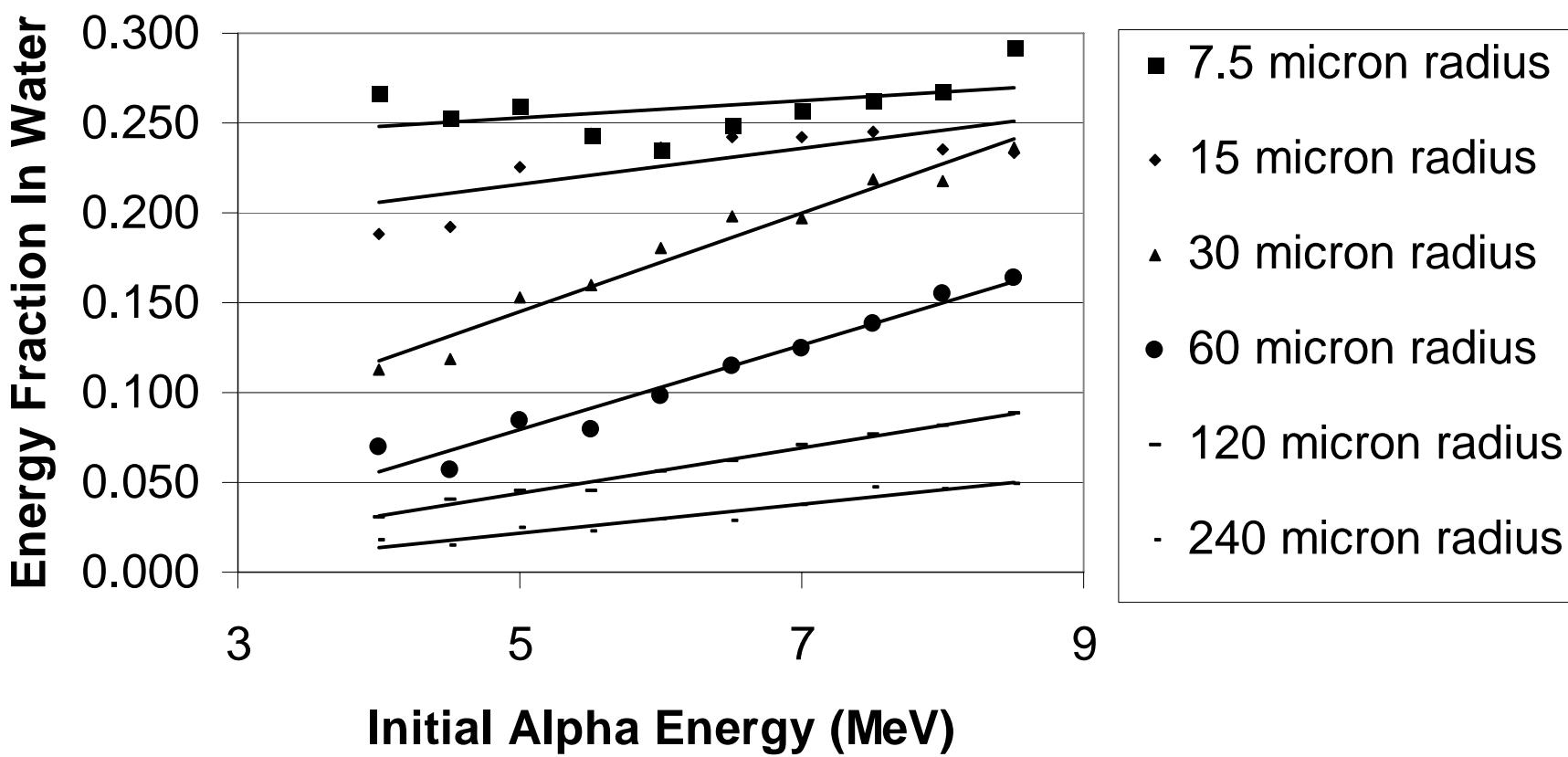
Objective

- **Provide systematic dosimetry model for predicting dose to water from SNF:**
 - As a function of radiation type
 - As a function of radiation energy
 - As a function of distance from the fuel
 - For fuel dimensions ranging from pellets to grains
 - For different fuel-to-water ratios
 - For assumed average fuel composition at Yucca Mountain and as a function of decay
 - For a variety of simulated, experimental conditions

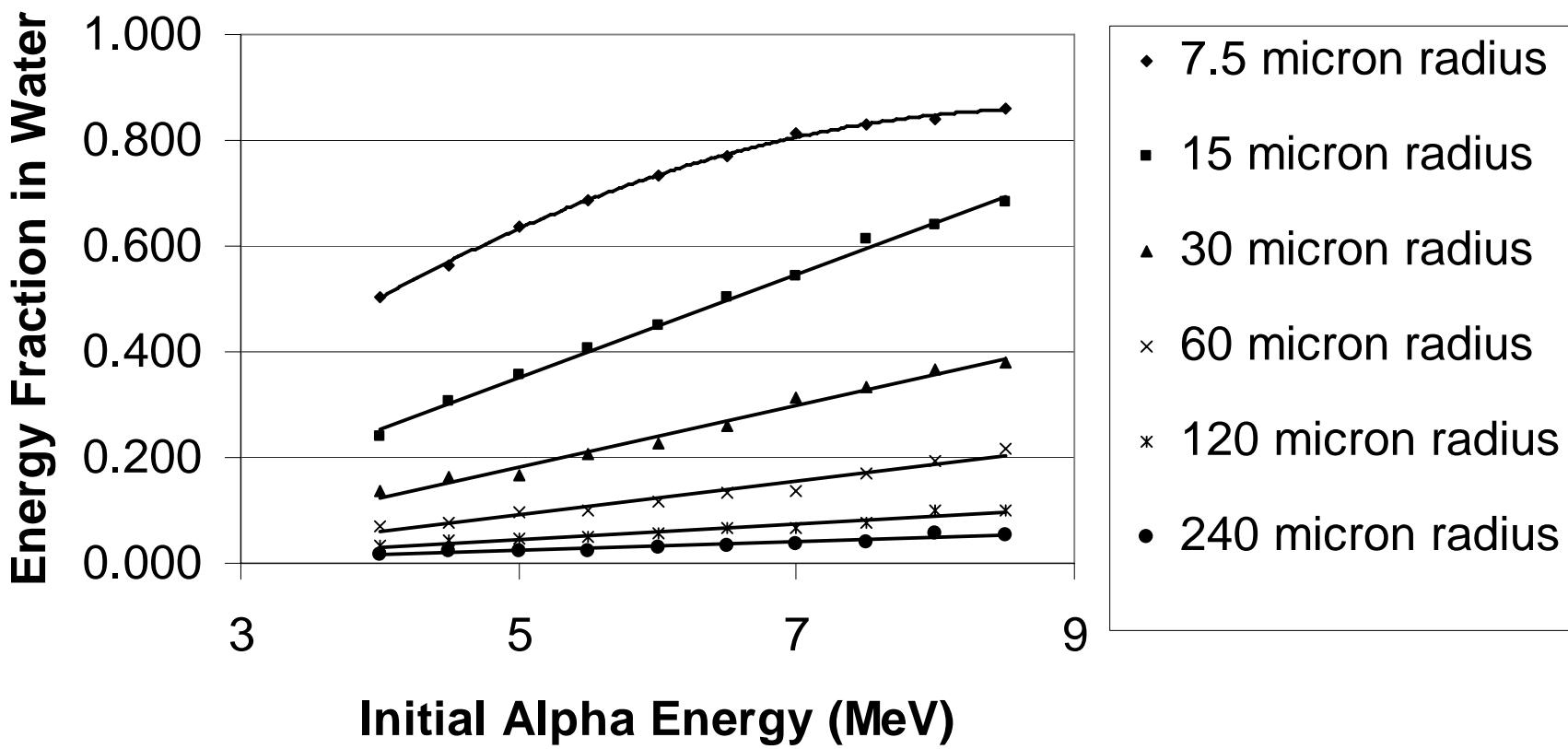
Radiation Transport Modeling

- Alpha Dosimetry
 - > Random walk (Monte Carlo) model implemented in Mathematica
- Gamma Dosimetry
 - > MCNP5
- Beta Dosimetry
 - > MCNP5 – multiple runs to simulate energy spectra
- Geometries
 - > 17 x 17 fuel pin, Westinghouse pellets
(0.392 cm radius x 0.940 cm)
 - > Grains from 7.5 μ to 240 μ in radius

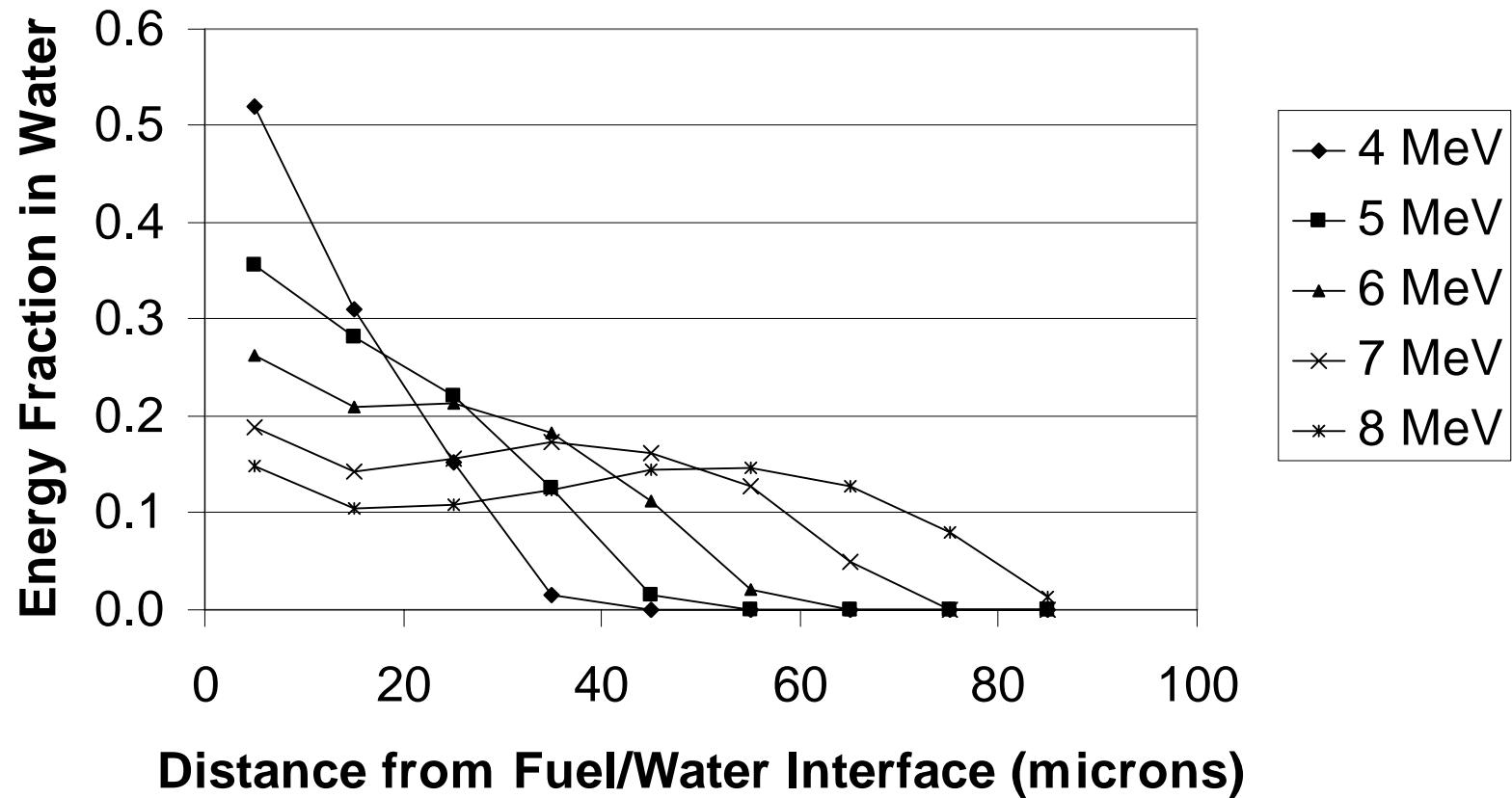
Alpha – Infinite Matrix of Grains



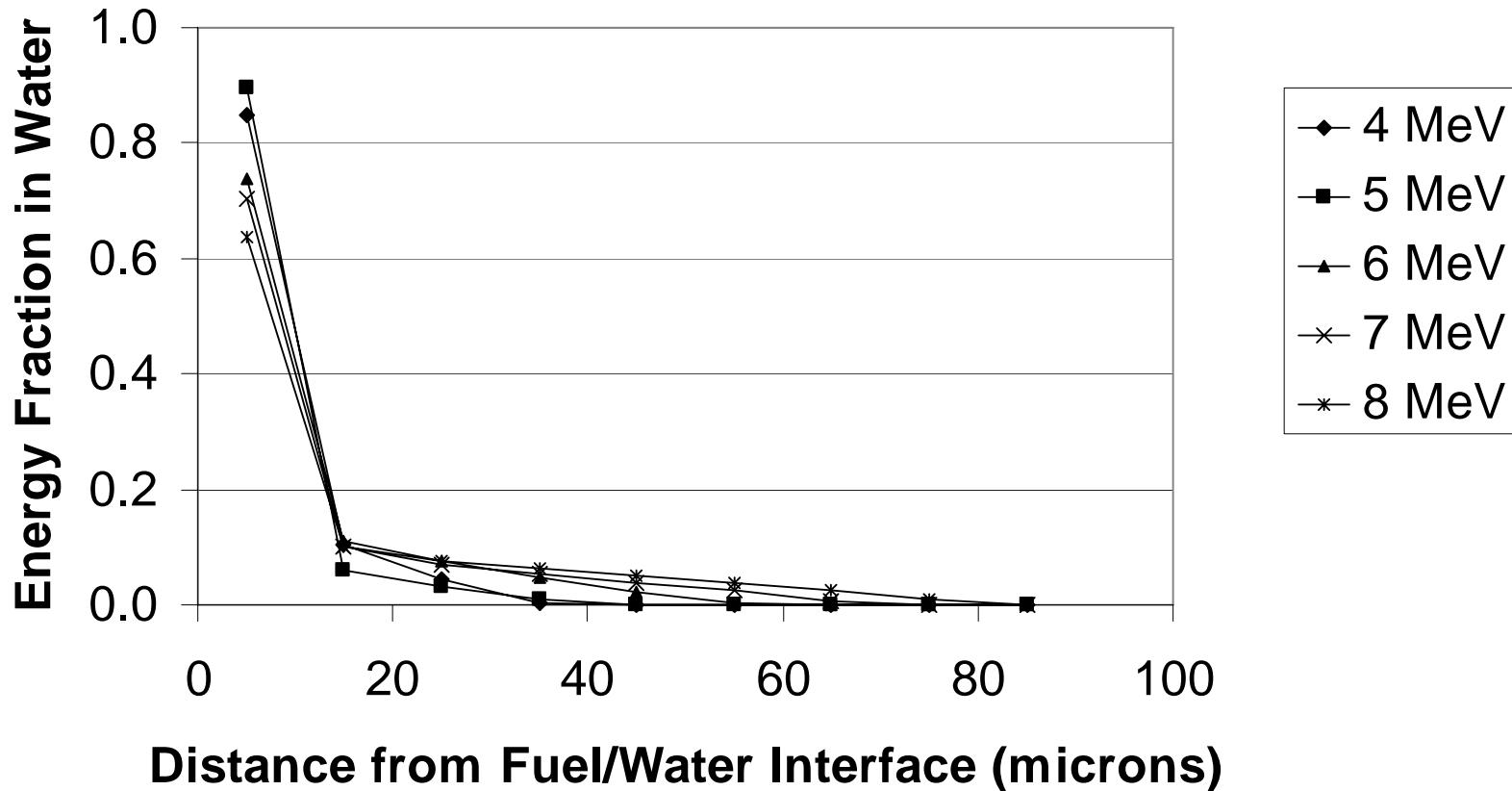
Alpha – Infinite Water



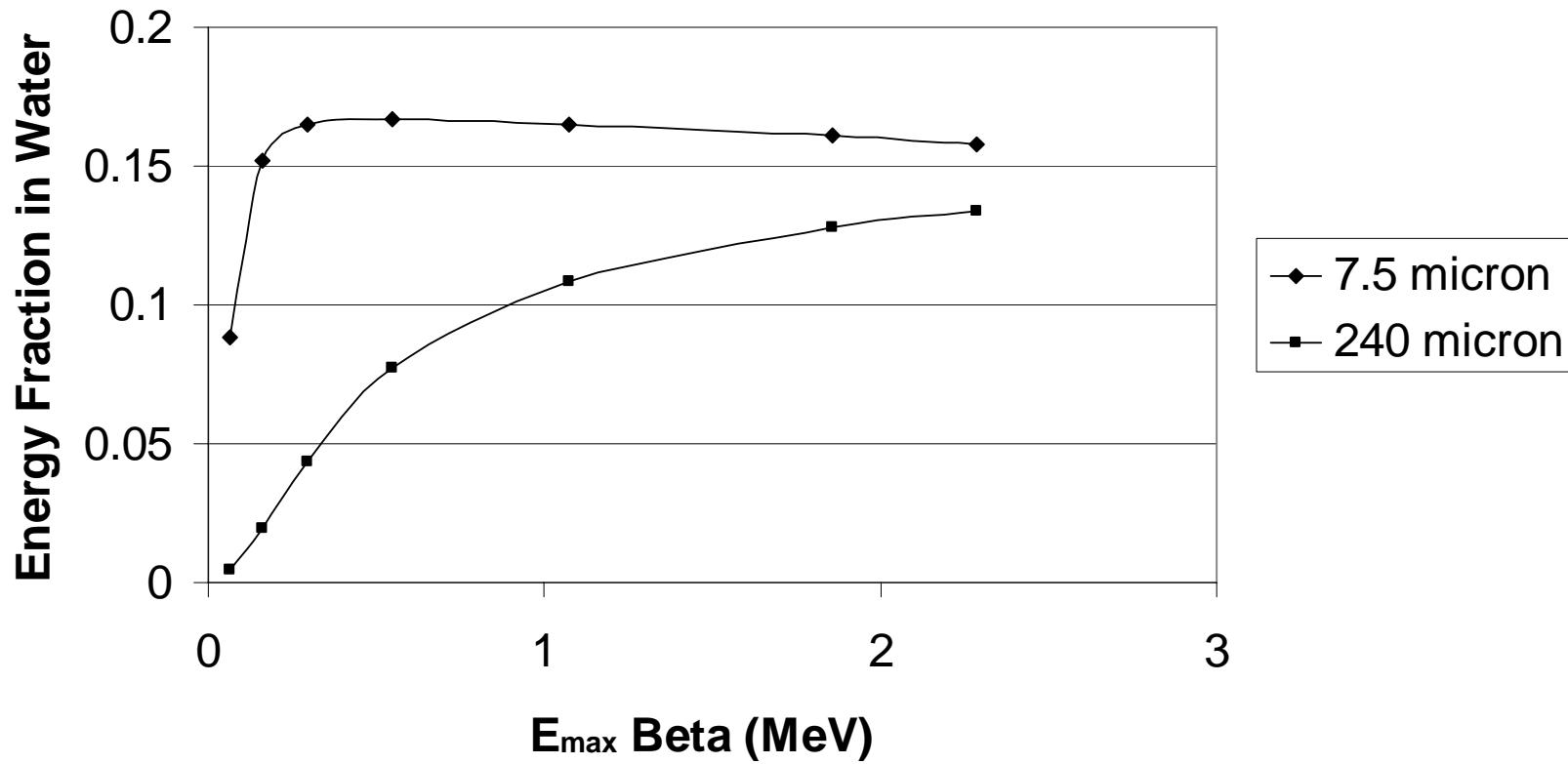
Alpha Spatial Energy Deposition – 7.5 μ Grain



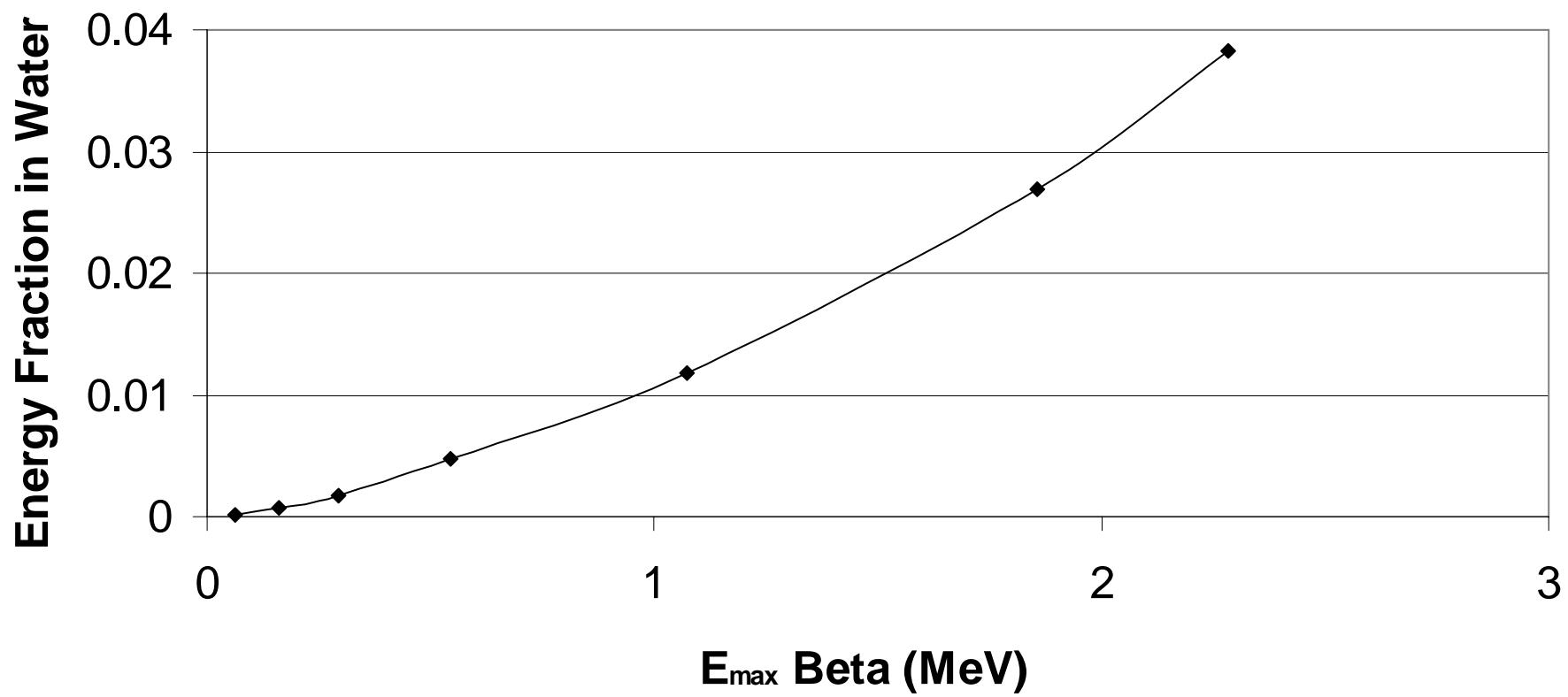
Alpha Spatial Energy Deposition – 30 μ Grain



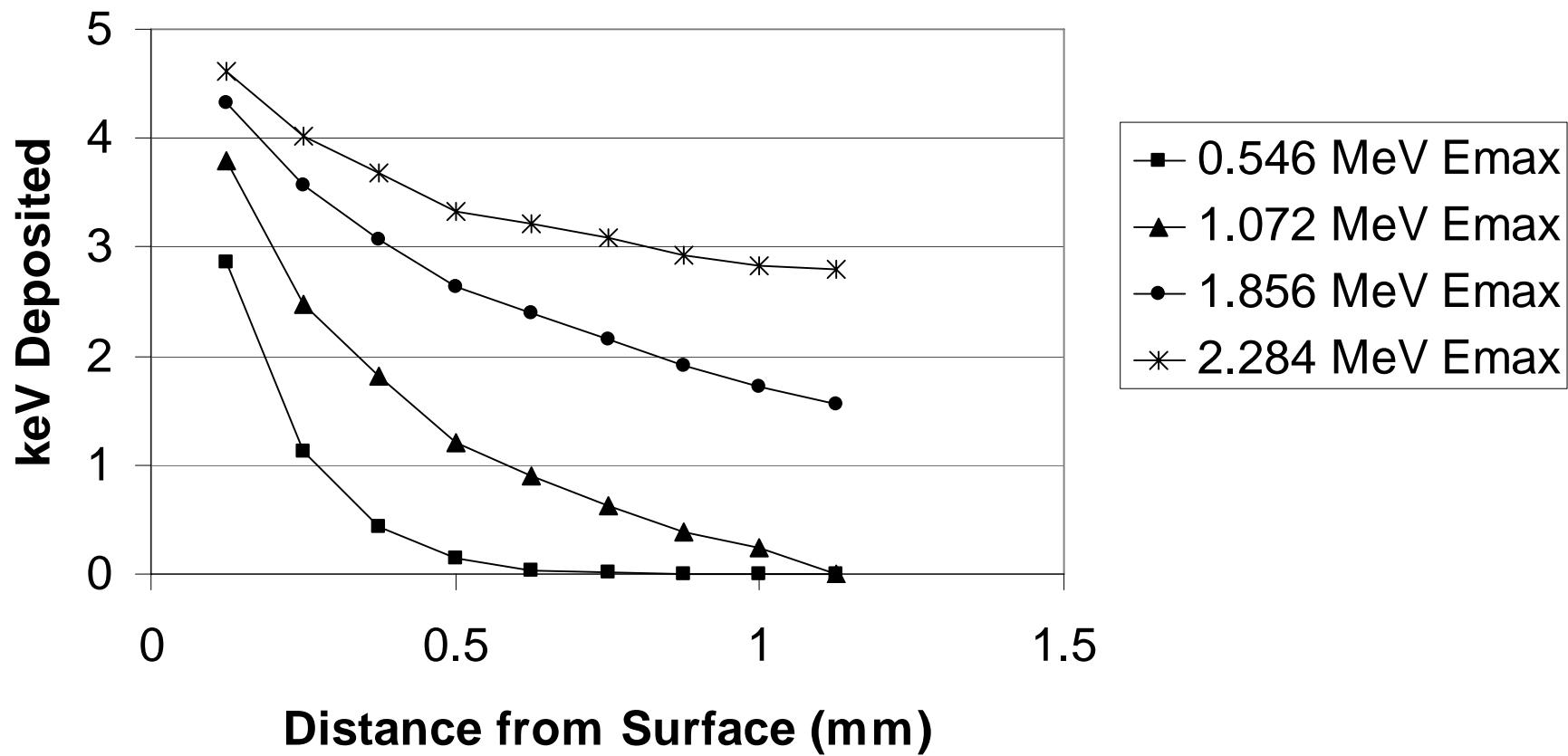
Beta – Infinite Matrix of Grains



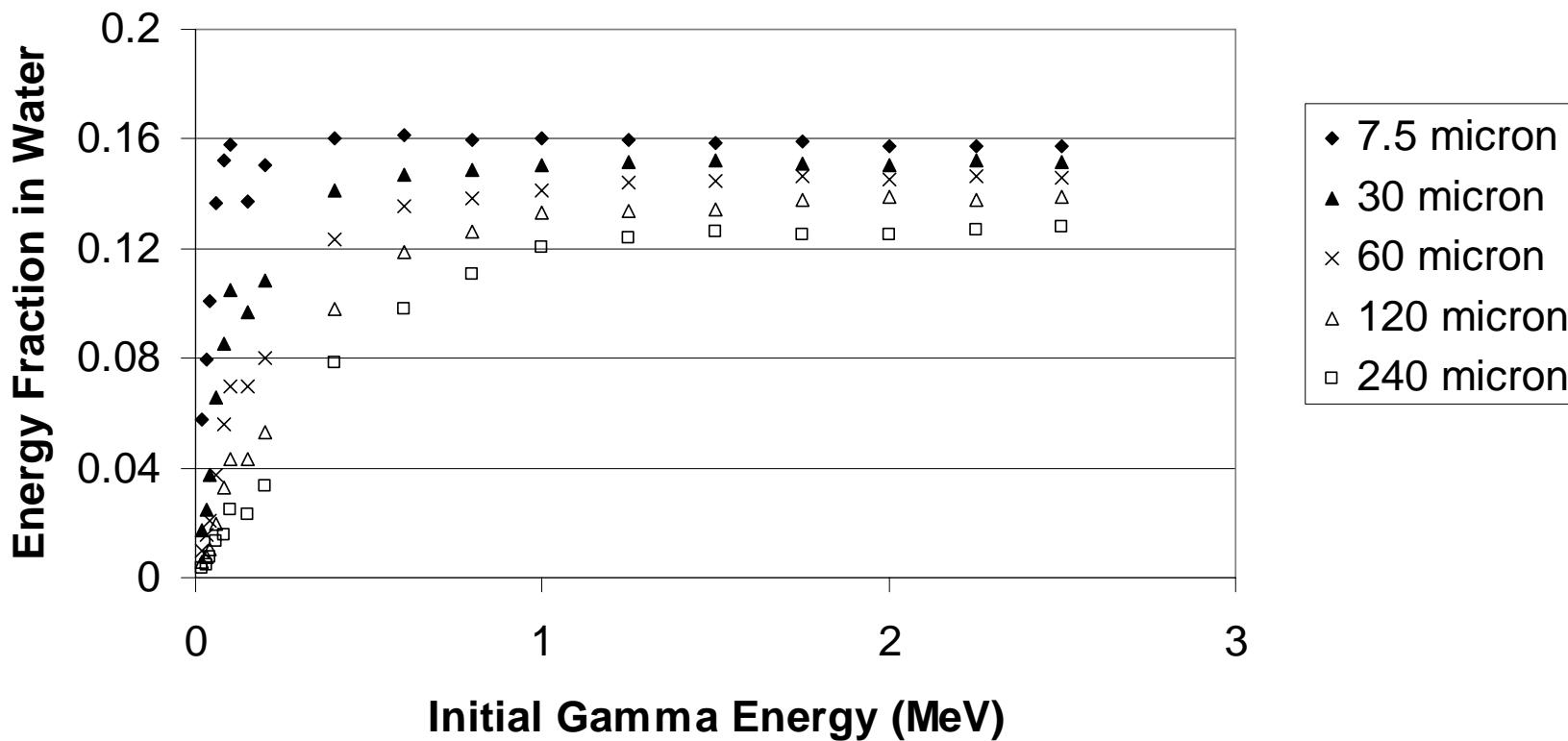
Beta Results – Fuel Pellet



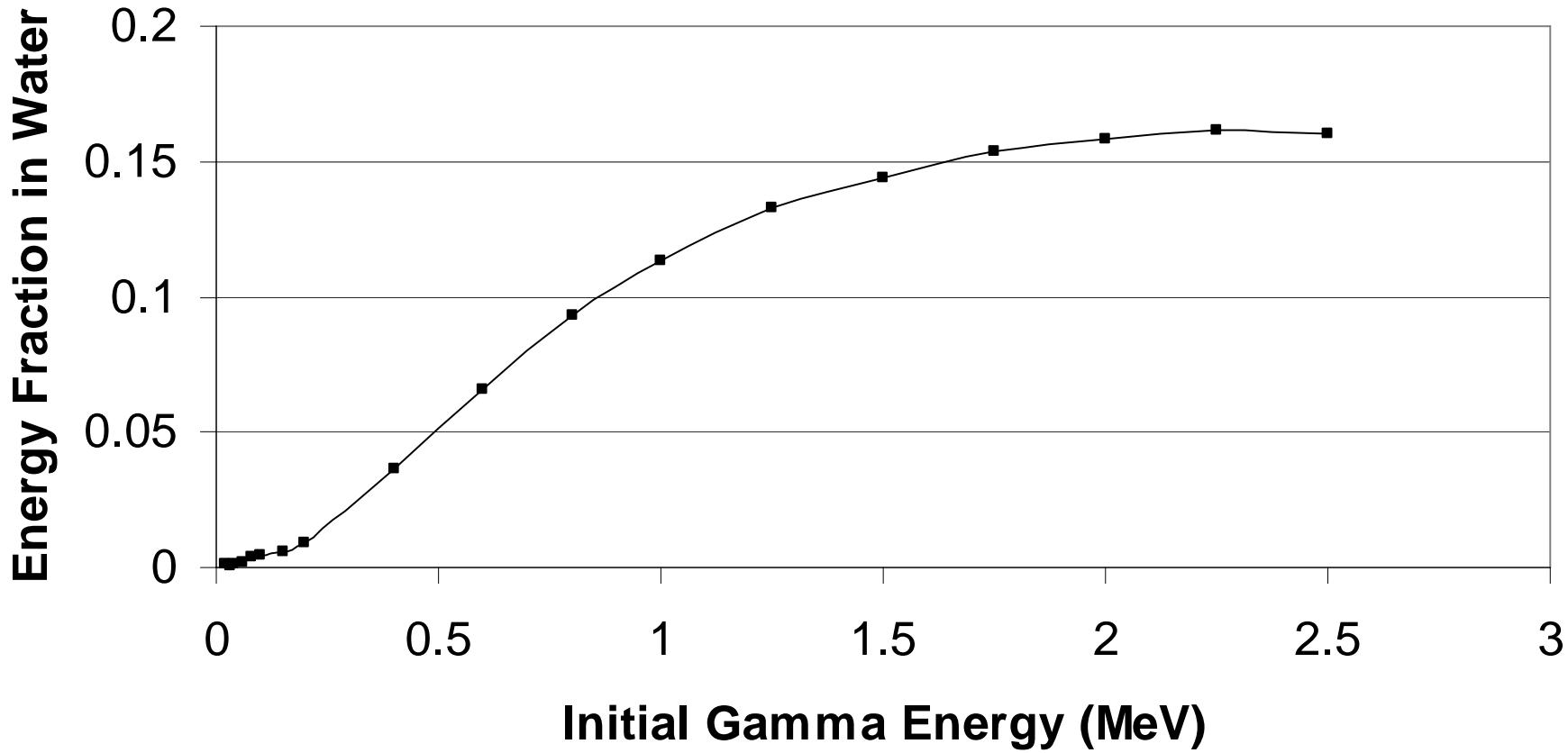
Beta Spatial Energy Deposition



Gamma – Infinite Matrix of Grains



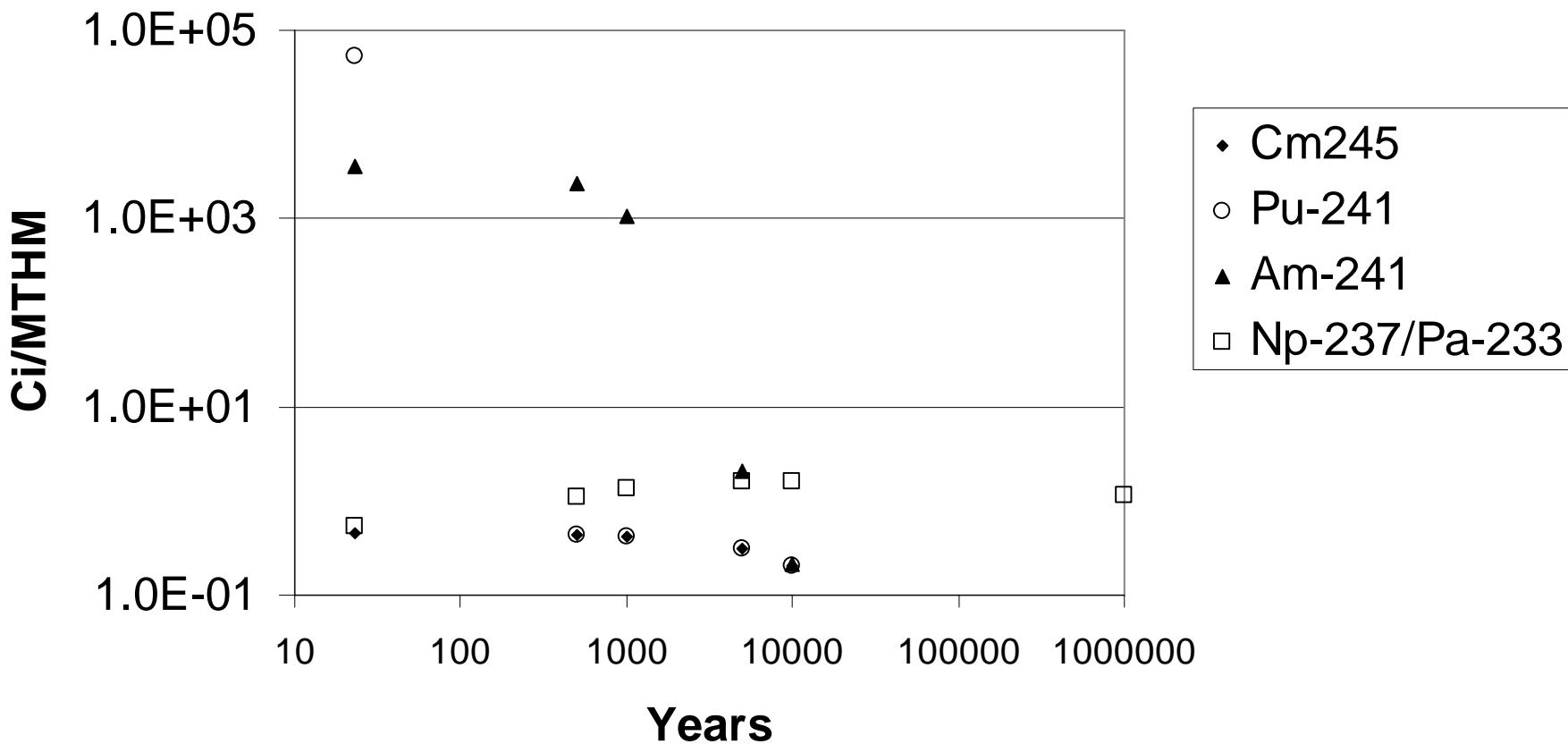
Gamma – Fuel Pellet



SNF Dose to Water Assumptions

- Used Yucca Mountain EIS Fuel for Initial Activities
 - PWR fuel
 - 3.75% enriched
 - Burned to 41,200 MWD/MT
 - Assumes initial 23 years of decay
- Solved coupled, first-order differential equations for transuranic activity vs decay using Polymath 6.0
- Modeled simple decay for fission products

Typical Decay Results



Spreadsheet of Emissions and Yields

Isotope Cm-245 Chain	Half Life	Q	Yield Alpha	Energy
Cm-245	8500	5.624	0.050	5.304
			0.932	5.362
Pu-241	14.35	0.022		
Am-241	432.2	5.638	0.016	5.388
			0.130	5.443
			0.845	5.485
Np-237	2.14E+06	4.959	0.062	4.639
			0.033	4.664
			0.080	4.766
			0.250	4.771
			0.470	4.788
			0.016	4.803
			0.025	4.817

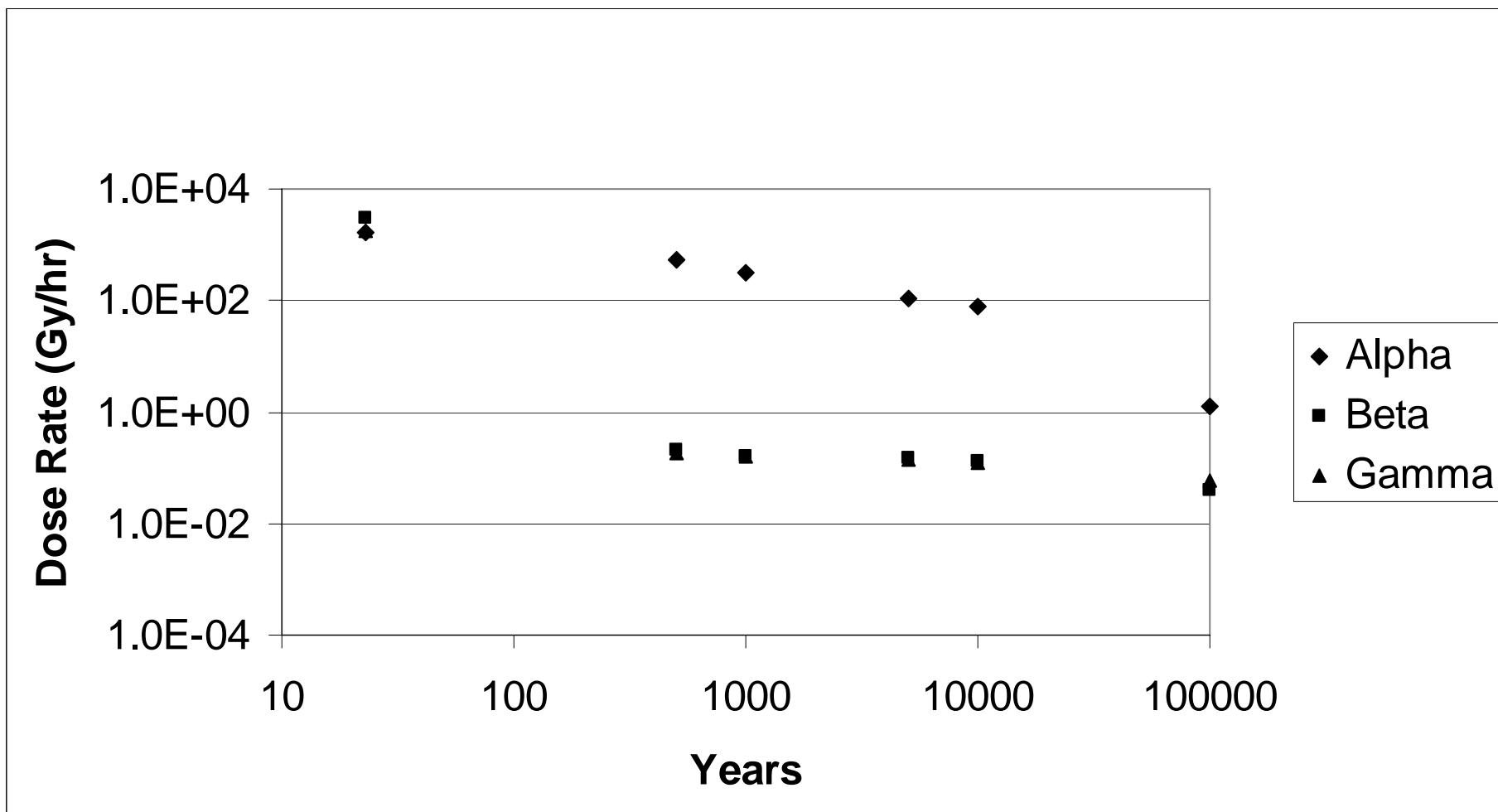
Dose Calculation

- **Combine**

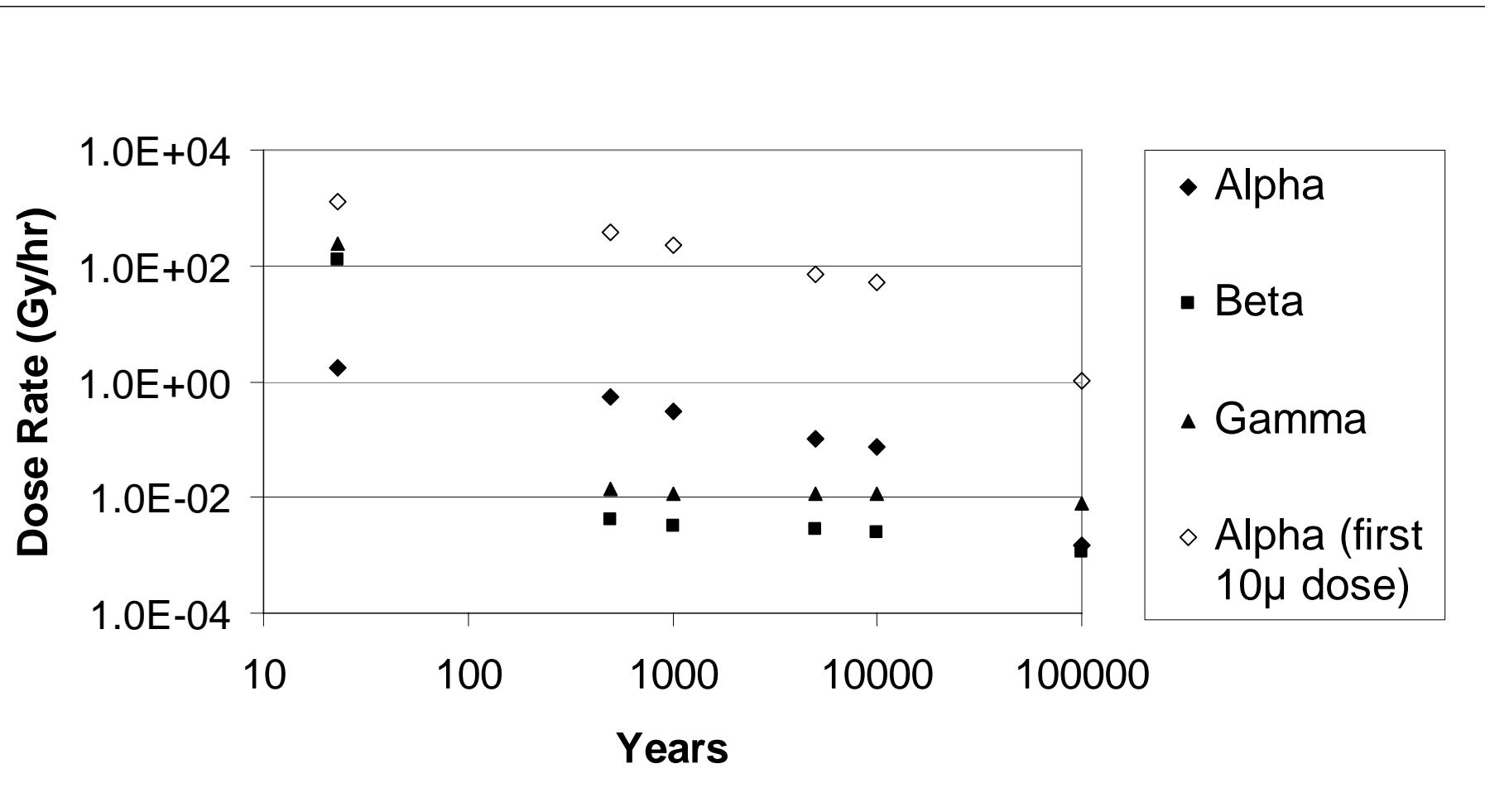
- › Types of emissions for all major isotopes in SNF
- › Yields and energy of emissions
- › Fraction of energy deposited in water for each emission and energy from transport calculations
- › vs. time

- **Results in dose to water**

Dose to Water for Infinite Array of 7.5 μ Grains



Dose to Water for Infinite Array of Pellets



Verification – Gamma Dose

	This Work*	BSC 2002**
~25 yrs decay	230 Gy/hr	400 Gy/hr
10,000 yrs decay	0.01 Gy/hr	0.03 Gy/hr

* 17x17 array fuel pellets, 3.75% enriched, burned to 41,200 MWD/MT.

** 15x15 array fuel pellets, 4% enriched, burned to 48,000 MWD/MT
("Gamma and Neutron Radiolysis in the 21-PWR Waste Package from
Ten to One Million Years," OOO-OOC-DSU0-00700-00-00A. Las
Vegas, Nevada: Bechtel SAIC Company, AC: MOL.20021004.0000,
2002).

Verification – Alpha Dose

This Work*	C. Jegou, et. al.**
1300 Gy/hr	2500 Gy/hr

- * 17x17 array fuel pellets, 3.75% enriched, burned to 41,200 MWD/MT, 23 years of decay.
- ** Simulated Fessenheim 2 reactor fuel pellets, 3.1% enriched, burned to 47,000 MWD/MT, 15 years of decay. Dose estimated from alpha flux and assuming 10 microns of range. (Jegou, C., B. Muzeau, V. Broudic, A. Poulesquen , D. Roudil, F. Jorion and C. Corbel, "Effect of alpha irradiation on UO_2 surface reactivity in aqueous media," Radiochim. Acta., **93**, 35–42, 2005).

Conclusions

- Comprehensive model has been developed
- Dose to water as a function of radiation type, energy, and fuel size has been calculated
- Model applied to Yucca Mountain SNF
- Model can easily predict dose to water for other experimental conditions
- These dose results, combined with radiolysis yields, can predict initial radiolytic products
- Combined with appropriate chemistry, the production of oxidants and reductants at the fuel/water interface can be predicted

Acknowledgement

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